Examining microstructures and mechanical properties of neutron and ion irradiated T91 and HT9 alloy

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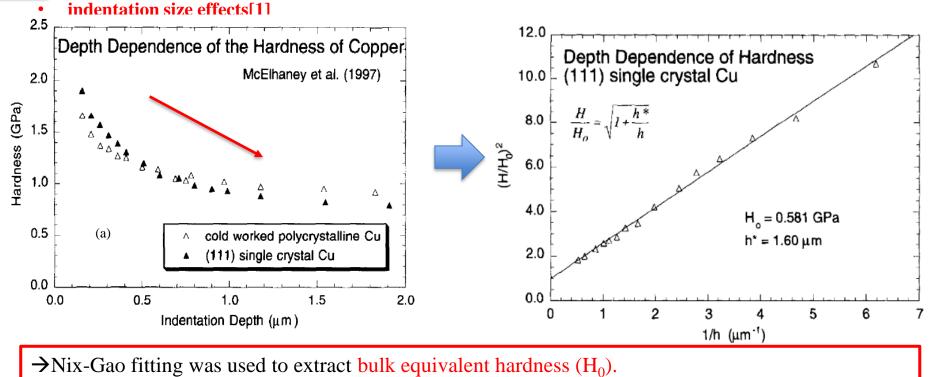


Introduction: Ion irradiation as a surrogate for reactor irradiation

 \rightarrow A successful simulation requires high similarities in both neutron-modified microstructures and neutron-induced **bulk properties** with those caused by ion irradiation.

Neutron irradiation		Ion irradiation			
Pros	Cons	Pros	Cons		
 →Uniform damage layer →bulk tensile testing is →high cost →long time needed to achieve →closer to real reactor environment →long time needed to achieve 		 →little or no radioactivity →fast and easy access to experimental facility →economic 	 →limited volume for mechanical testing →much higher damage rate than in the reactors 		
Tancila tast	Susby, etc. H _v Vickers test		H_n \mathbf{V} Nanoindentation		

Introduction: Nanoindentation background

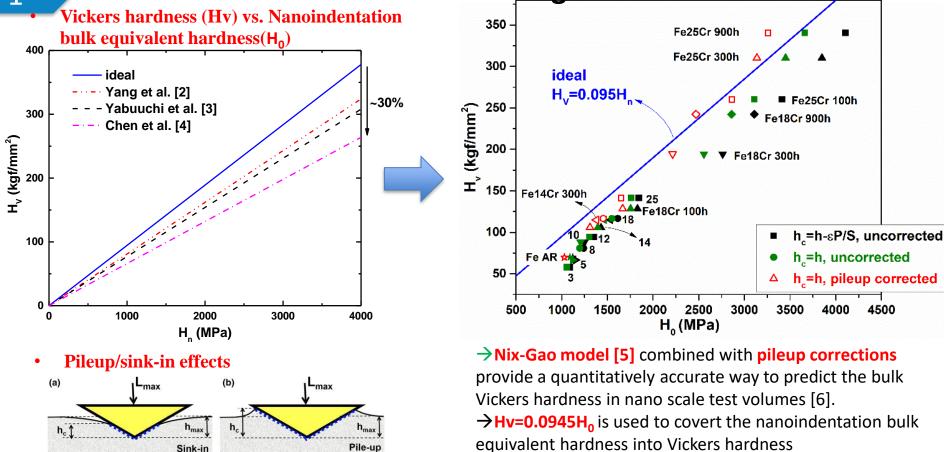


[1]K. W. McElhaney et al. JMR. 1998[2]Yang et al. JNM.2020

[3]Yabuuchi et al. JNM.2014 [4]Chen et al. Acta. Mater. 2020



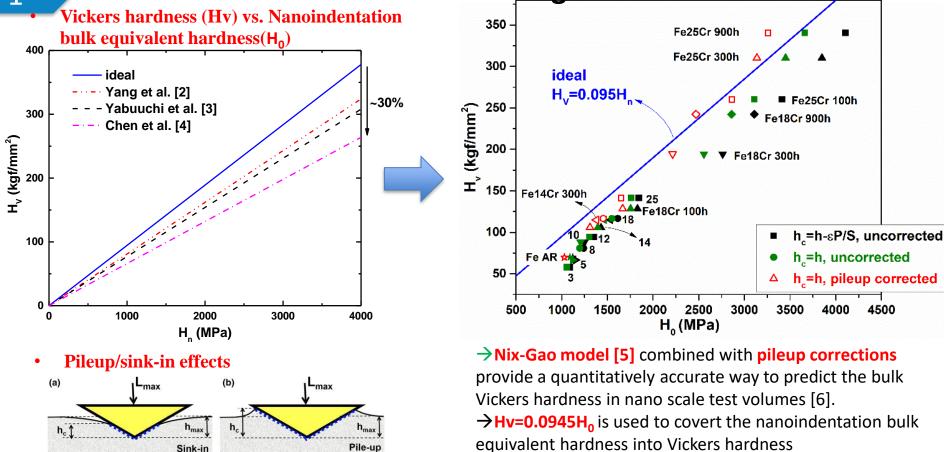
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[5] William D.Nix et al. J. Mech. Phys. Solid, Vol. 46, No. 3, pp. 41 I-425. 1998
[6] P. Zhu et al. Toward accurate evaluation of bulk hardness from nanoindentation testing at low indent depth. Material & Design <u>213</u> (2022), 110317

Introduction: Nanoindentation background



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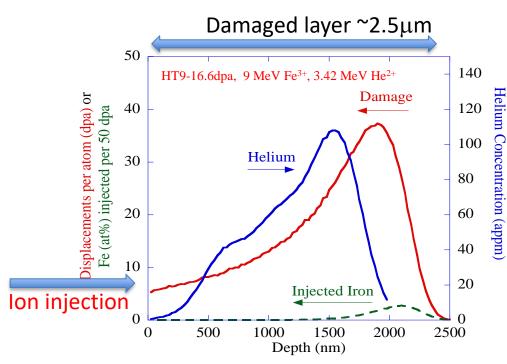
Sample information

9 MeV	Fe ³⁺ .	3.42	MeV	He ²⁺
5	• • • •			

Dual ion	Irradiation T(°C)	dpa	He appm He/dpa		
T91	445	16.6	72	4.3	
	520	16.6	72	4.3	
HT9	445	72	309	4.3	
	520	16.6	72	4.3	

BOR-60

Neutron	Irradiation T(°C)	dpa
T91	369	16
191	453	19
	450	32
HT9	453	19



- \rightarrow The total irradiation depth for 9MeV irradiation is about 2.5 um.
- → The shallow damage layers inhibit the bulk mechanical testing methods such as tensile and Vickers hardness tests.



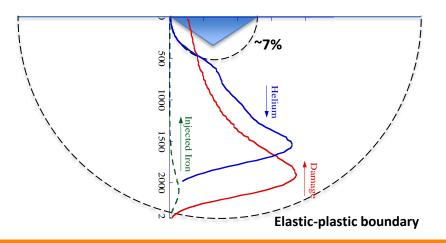
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Results: Nanoindentation testing on dual-ion irradiated samples

 \rightarrow the plastic zone generally expands 5-10 times the indentation depth.

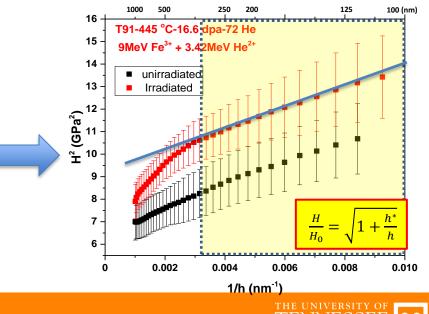
3

 \rightarrow only data from first several hundreds of depth is available to avoid substrate effect.



Strain extension under indenter

 \rightarrow H² vs. 1/h curve for typical ion irradiated sample shows inflection around 300 nm, which infers the effect of un-irradiated substrate layer.



L.E. Samuels & T.O. Mulhearn, J. Mech. Phys. Sol. 5 (1957) 125

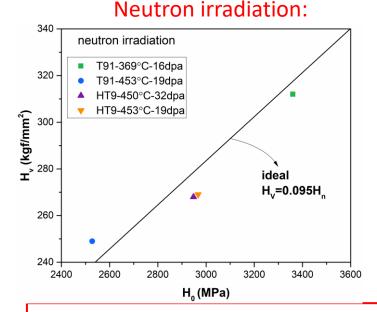
Results: Nanoindentation and Vickers on T91 and HT9

 \rightarrow The test protocol developed on thermally aged FeCr alloys was used to extract nanoindentation bulk equivalent hardness. Since the pileup measurement on neutron samples is not available, the same amount of pileup effect was estimated with the ion irradiated samples.

Dual ion irradiation:

3

Sample	temperature	dose	H ₀ (pileup corrected)	pileup correction	Hv=94.5H ₀	Measured Hv
	°C	dpa	(GPa)		(kgf/mm ²)	(kgf/mm²)
T91	445	16.6	3.02	8%	286	-
		control	2.55	12%	241	247
	520	16.6	2.34	10%	221	
		control	2.34	9%	221	235
HT9	445	72	3.61	5%	341	-
		control	2.93	7%	277	-
	520	16.6	2.74	6%	259	-
		control	-	-	-	-



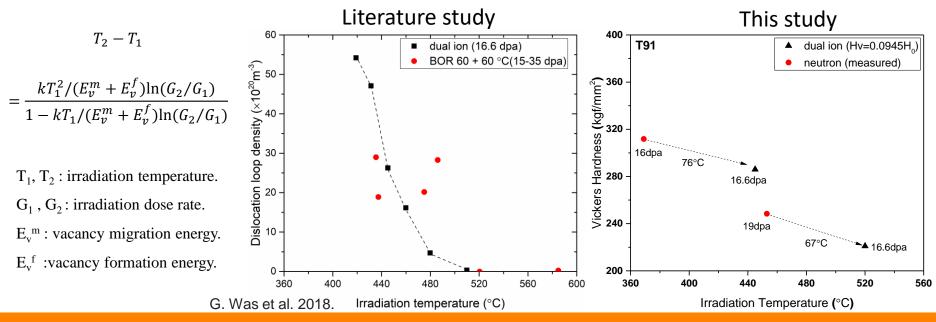
→Good agreement is achieved between nanoindentation and Vickers testing results after applying the nanoindentation test protocol on neutron irradiated samples.

Results: Temp. shift to account for dose rate

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 \rightarrow A temperature shift is needed to account for the dose rate difference between ion and neutron irradiation.

→The temperature shift seems to result in both a qualitative and quantitative match of the dislocation loops, and seems to work well in the mechanical properties of neutron and dual ion irradiated samples.



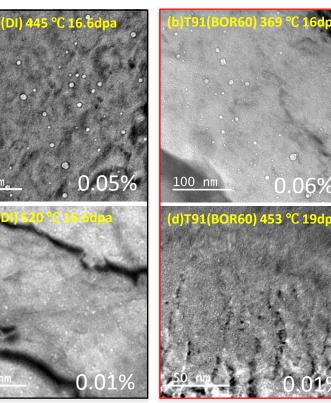
TENNESSEE KNOXVILLE

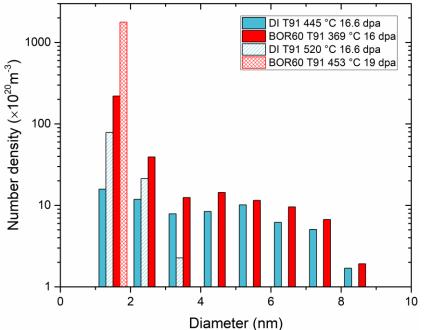
Results: Cavities in irradiated T91

Neutron

Dual ion

3



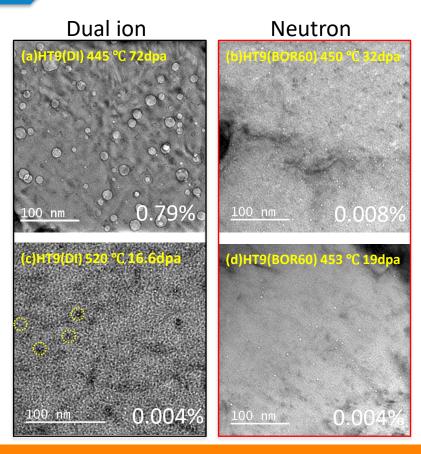


 \rightarrow Lower temperature generated a wider range of cavity size for both dual-ion and neutron irradiation.

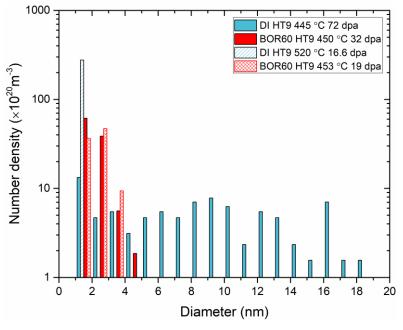
 \rightarrow The void swelling matches well for dual ion and neutron samples with a ~70 °C temperature shift.



Results: Cavities in irradiated HT9



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 \rightarrow No clear bimodal distribution was observed. At higher temperatures, the cavities tended to be limited to small sizes.

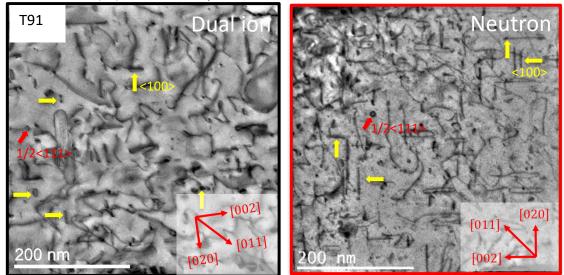
 \rightarrow The void swelling matches well for dual ion at 520 °C and neutron samples at 453°C.



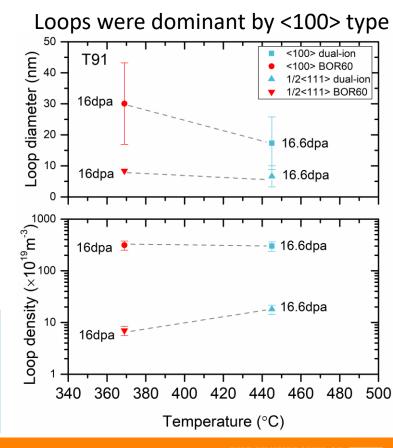
3 Results: Dislocation loops in irradiated T91

T91(DI) 445 °C 16.6dpa

T91(BOR60) 369 °C 16dpa



→Shifting the temperature of BOR 60 irradiated T91 about ~70 °C results in a quantitative match of dislocation loops (both <100> and $\frac{1}{2}$ <111> loops) with dual ion irradiated T91 sample. The slight discrepancy may come from the non homogenous distribution of the dislocation loops.

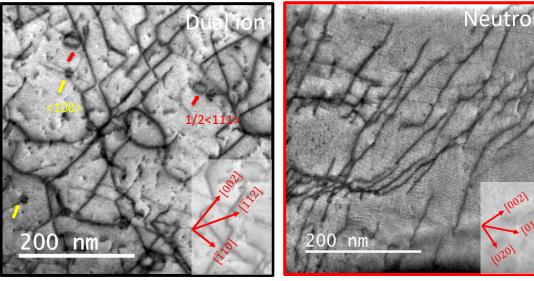


Results: Dislocation loops in irradiated HT9

HT9(BOR60) 453 °C 19dpa

HT9(DI) 520 °C 16.6dpa

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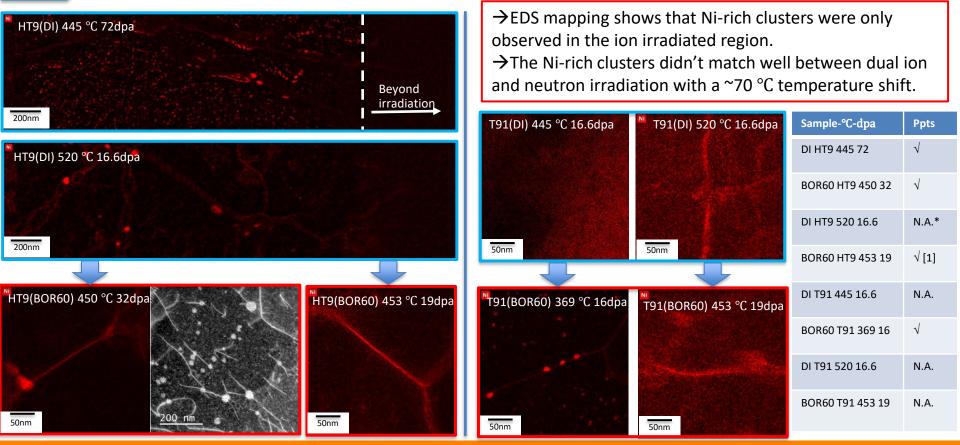


→ However, dislocation loops were only observed in dual ion irradiated samples for HT9, in a very low density.

Material	Temperature	Dose	<100>	Іоор	1/2<111> la	Dislocation line	
	°C	dpa	Average diameter Number densi		Average diameter	Density	Density
			(nm)	(×10 ²¹ m ⁻³)	(nm)	(×10 ²¹ m⁻³)	(×10 ¹⁴ m ⁻²)
DI-HT9	445	72	34.3 ± 13.1	1.6 ± 0.3	-	-	4.4 ± 0.9
	520	16.6	10.4 ± 2.4 0.5 ± 0.1		31.0 ± 11.3	0.4 ± 0.1	4.2 ± 1.1



Results: Precipitates & Radiation Induced Segregation



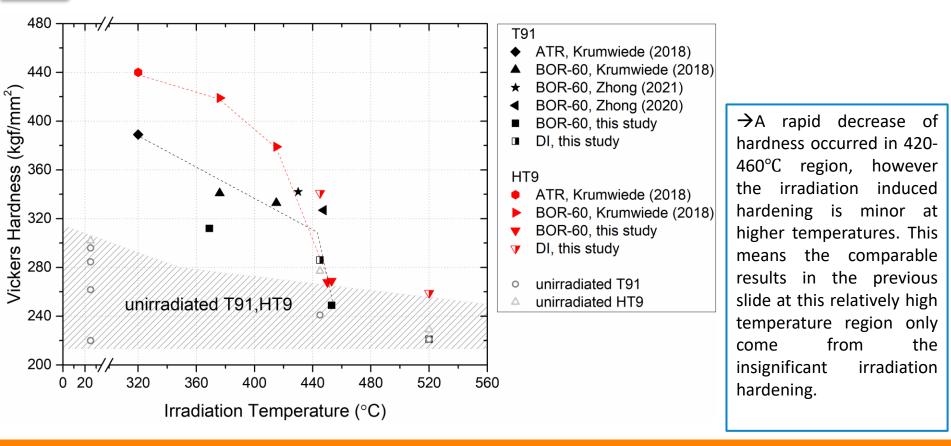


*only counted Ni-clusters in the matrix

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[1] C. Zheng et al. (2020) 151845

Discussion: Mechanical properties





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Discussion: Strengthening model

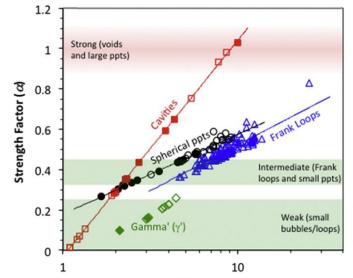
Dispersed barrier hardening(DBH): $\Delta \sigma = \alpha M \mu b \sqrt{Nd}$

- $\Delta \sigma$ ----the change in yield strength,
- M ----Taylor factor (3.06 for equiaxed BCC polycrystalline metals)
- α ----obstacle strength factor
- *b* ----Burgers vector (a/2<111> in current study where a=0.248 nm)
- d ----defect diameter
- N---- is the defect number density.

Calculate each of the strengthening contribution

Hardening contribution in this study:

- Pure Fe: σ_{Fe}
- \bullet Solute solution: σ_{ss}
- Cavities (diameter<2nm and >2nm): σ_c
- Carbides: $\sigma_{carbides}$
- \bullet Ni/Si-clusters (large diameter): σ_{Ni}
- Dislocation loops: σ_{loop}
- Dislocation lines: σ_{line}
- \bullet Grain boundary: σ_{GB}



Size (d, nm)

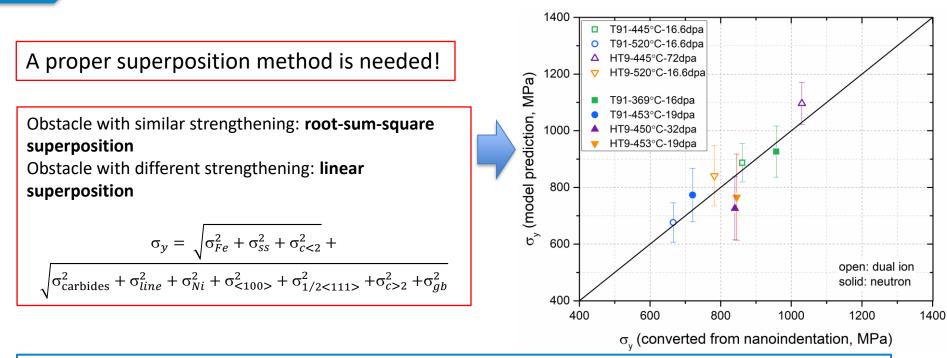
sample	Temp.	Dose	Disl	Disl. loops		Carbides	Cav	vities
	°C	dpa	<100>	1/2<111>	ppts.		d<2nm	d>2nm
DI-T91	445	16.6	0.50	0.33	-	0.74	0.21	0.72
	520	16.6	-	-	-	0.76	0.17	0.40
DI-HT9	445	72	0.64	-	0.48	0.91	0.13	1
	520	16.6	0.40	0.58		0.99	0.13	
N-T91	369	16	0.63	0.36	0.38	0.91	0.12	0.65
	453	19	-	-	-	0.74	-	-
N-HT9	450	32	-	-	0.67	0.83	0.19	0.43
	453	19	-	-	0.69	1	0.22	0.42



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Discussion: Microstructure vs. mechanical properties

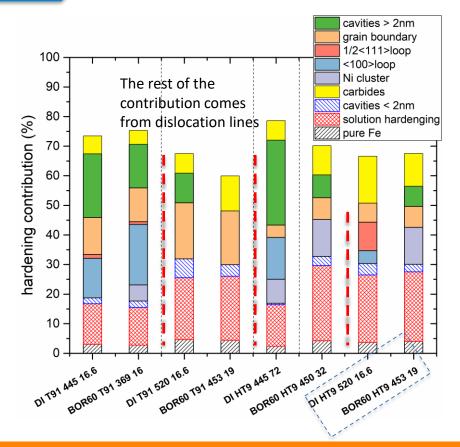
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 \rightarrow The good quantitative agreement between the nanoindentation-tested mechanical properties and microstructure-predicted bulk properties indicates that nanoindentation can accurately predict bulk mechanical strength (within ~10% accuracy) in nano scale volumes.



Discussion: Microstructure vs. mechanical properties



Δ

- → This comparison provides another perspective of the problem of using a simplistic 70 °C temperature shift for the neutron vs. ion irradiation conditions.
- → The corresponding neutron and ion irradiation pairs of hardening data did not share the similar percentage of the individual strengthening contribution, which is a reflection of their inconsistent microstructures especially for HT9 alloys.



Conclusions

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Ion irradiation as a surrogate of neutron irradiation

- (1) Nanoindentation with pileup correction is important and effective for obtaining quantitatively accurate bulk equivalent hardness values for ion and neutron irradiated materials.
- (2) The temperature shift theory works generally well for the cavities and dislocation loops observed in T91 following ion and neutron irradiations, but it is not suitable for the match of precipitates. Especially for the HT9 material, both dislocation loop and precipitate evolution didn't follow the calculated temperature shift.
- (3) The comparable nanoindentation results from dual-ion and neutron irradiated T91 and HT9 at elevated temperatures (>450 °C) may be largely due to the minimal irradiation hardening at elevated temperatures. The discrepancy in the microstructures indicates that more complex modeling should be used to link ion vs neutron irradiation microstructures and hardening.



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