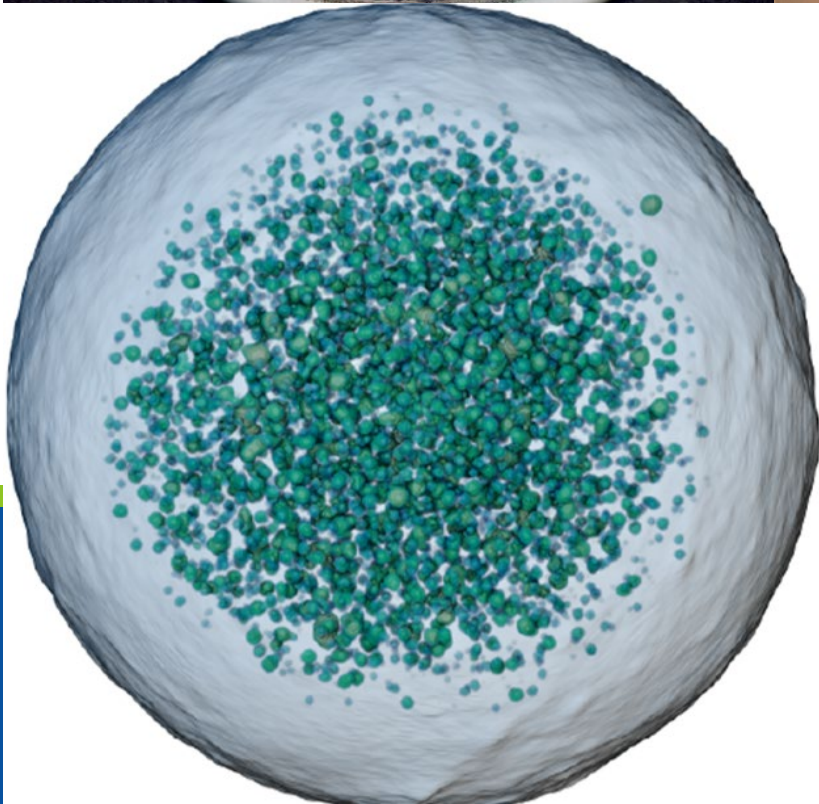


**Daniel Murray**  
**Sr. Manager**  
**Materials Characterization**  
**Department**



# Fundamentals of Post-Irradiation Examination

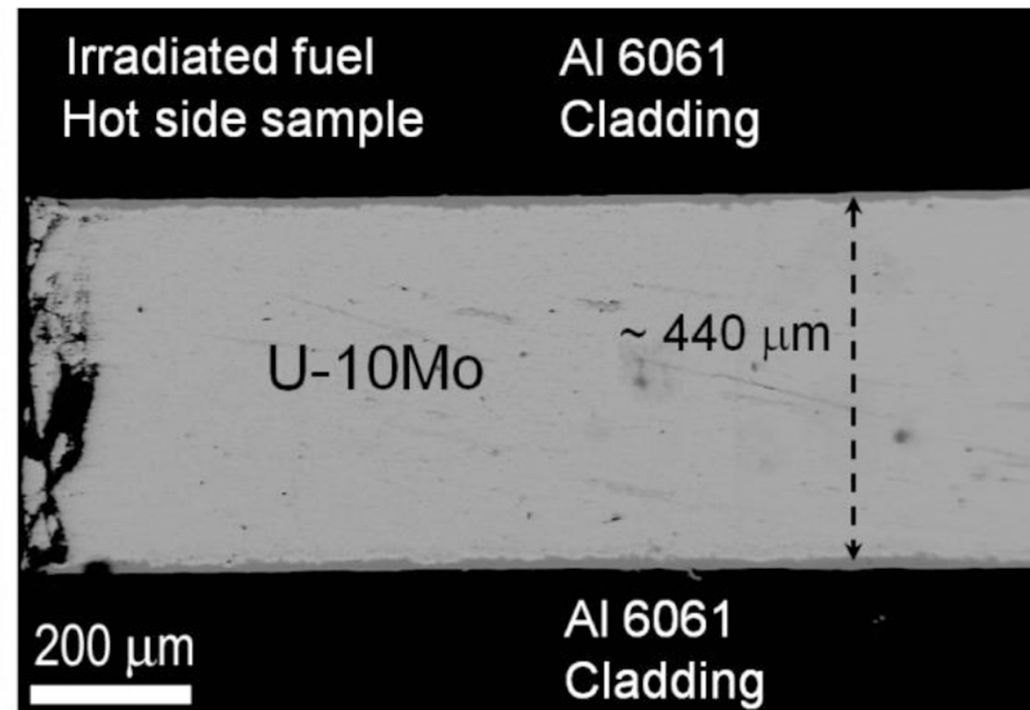
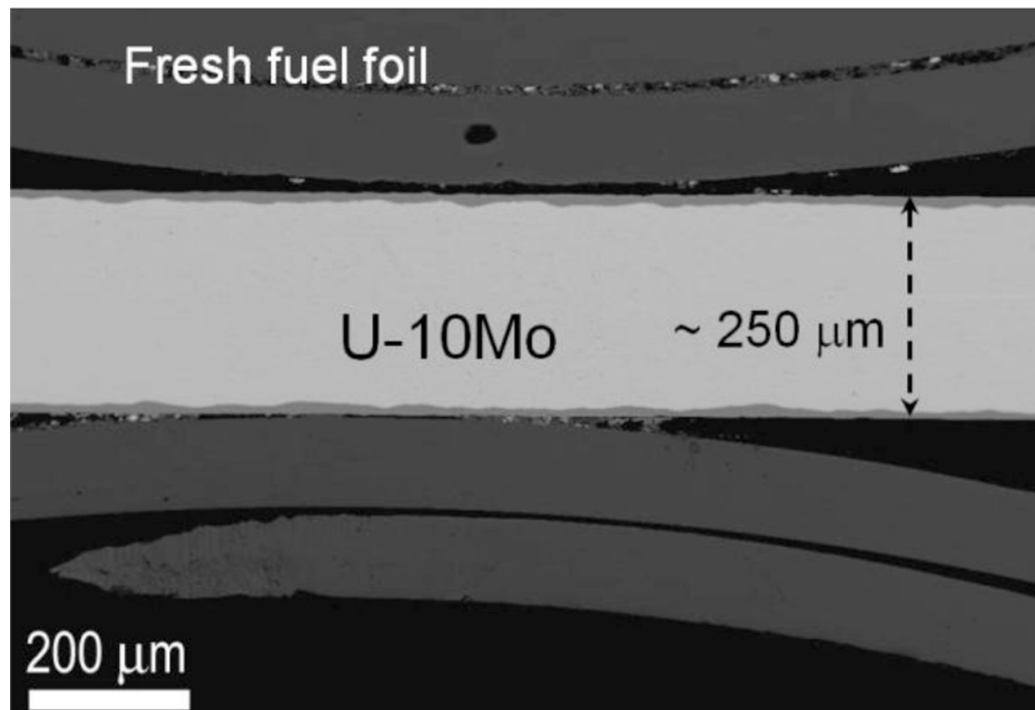
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INL/MIS-25-86017

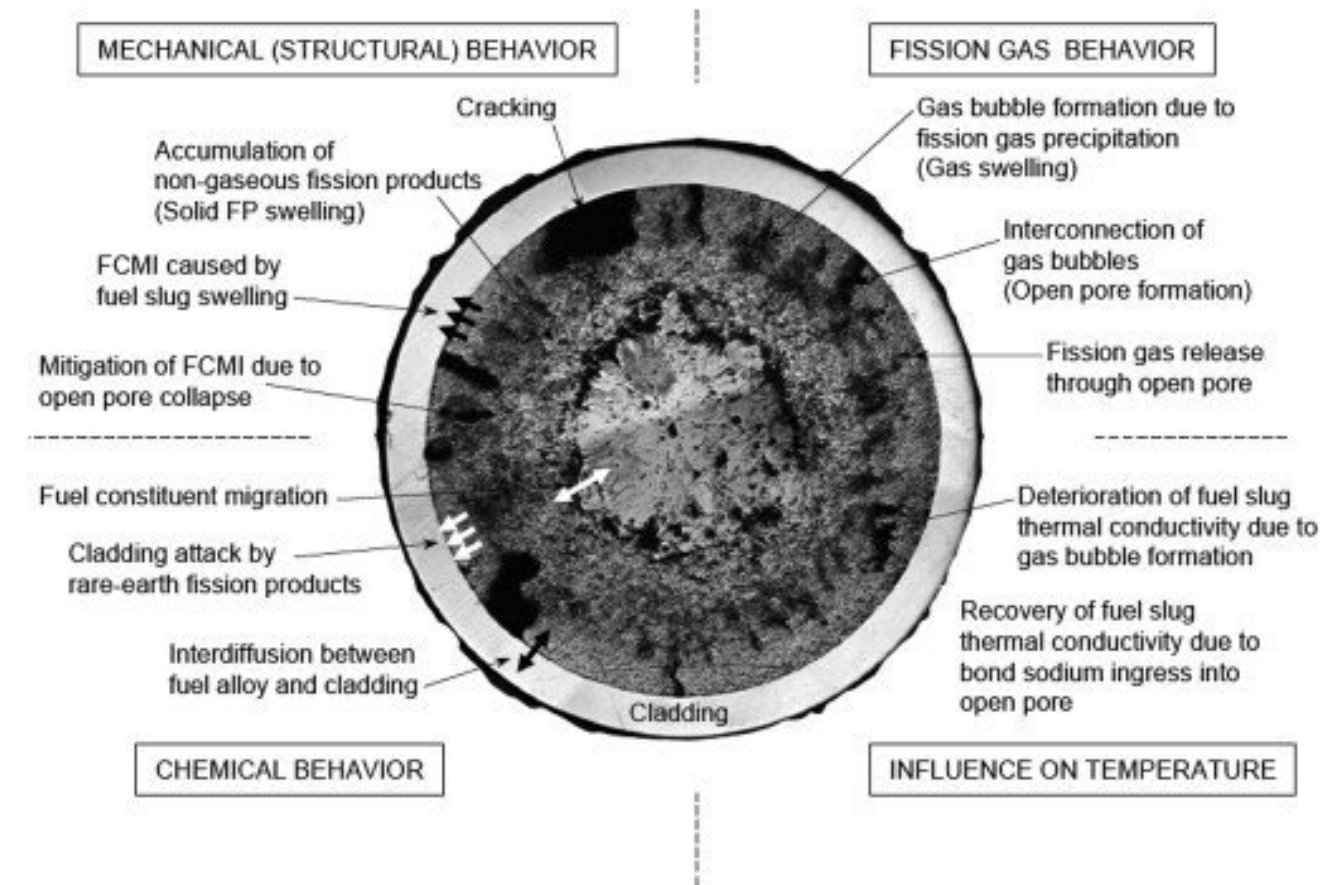
# What is Post-Irradiation Examination (PIE)?

Examinations conducted on irradiated material to understand physio-chemical changes that can be related back to in-reactor performance

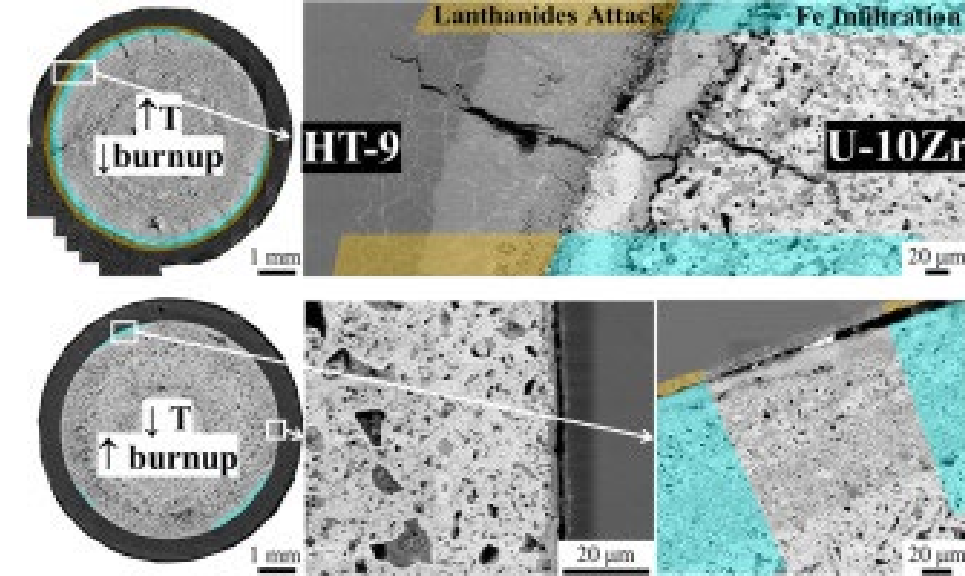




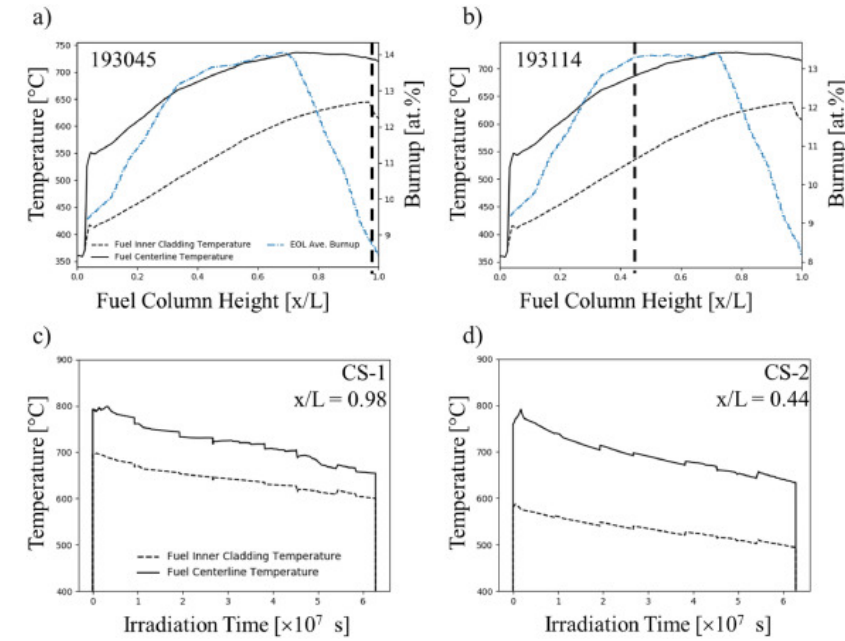
# Nuclear fuel performance



Experimental



Modelling



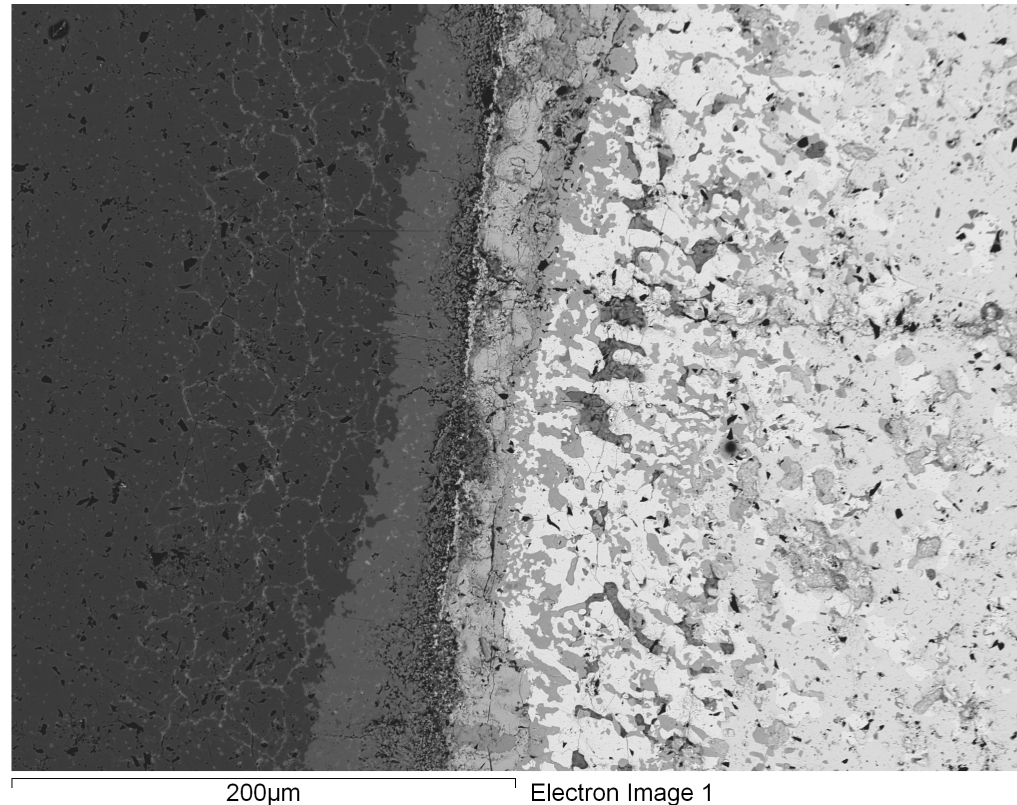
T. Sofu, Nuclear Engineering and Technology (2015), doi:10.1016/j.net.2015.03.004

D. Salvato et al., Journal of Nuclear Materials (2024), doi:10.1016/j.jnucmat.2024.154928.

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# Why is PIE Important?

PIE provides feedback to the fuel/materials designers, fabricators, reactor operators and fuel model developers on the performance of fuel and structural components.

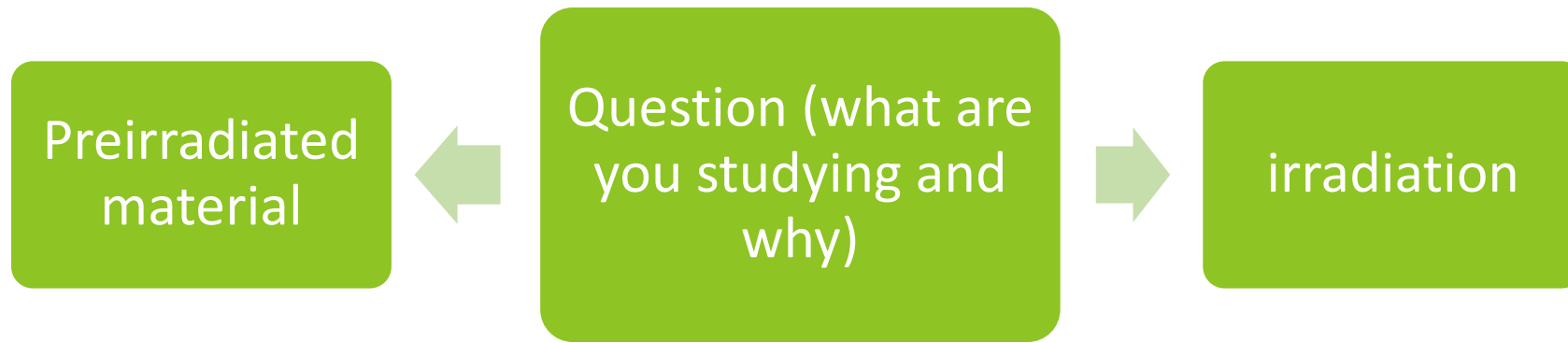


Fuel Cladding Chemical Interaction between HT-9 and U-Zr fuel

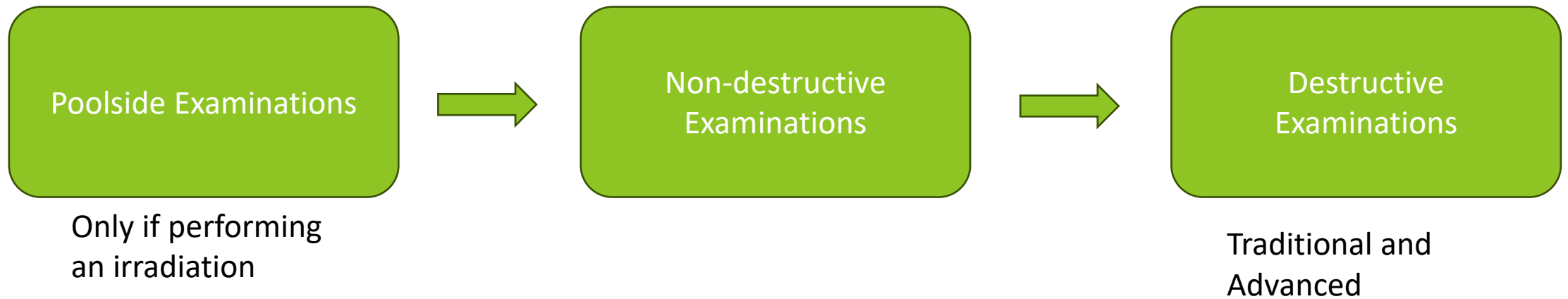


# Approaches to PIE

- Starts with scientific question (like any research)



# Process Flow

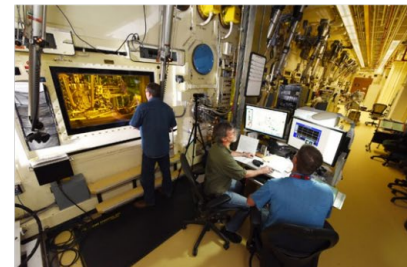
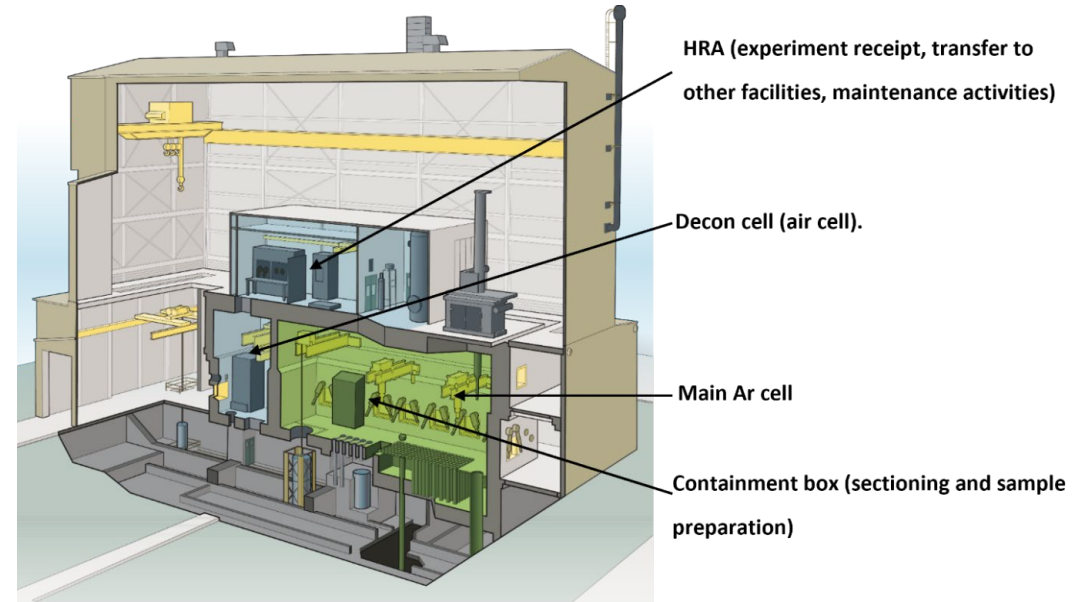




# PIE Facilities

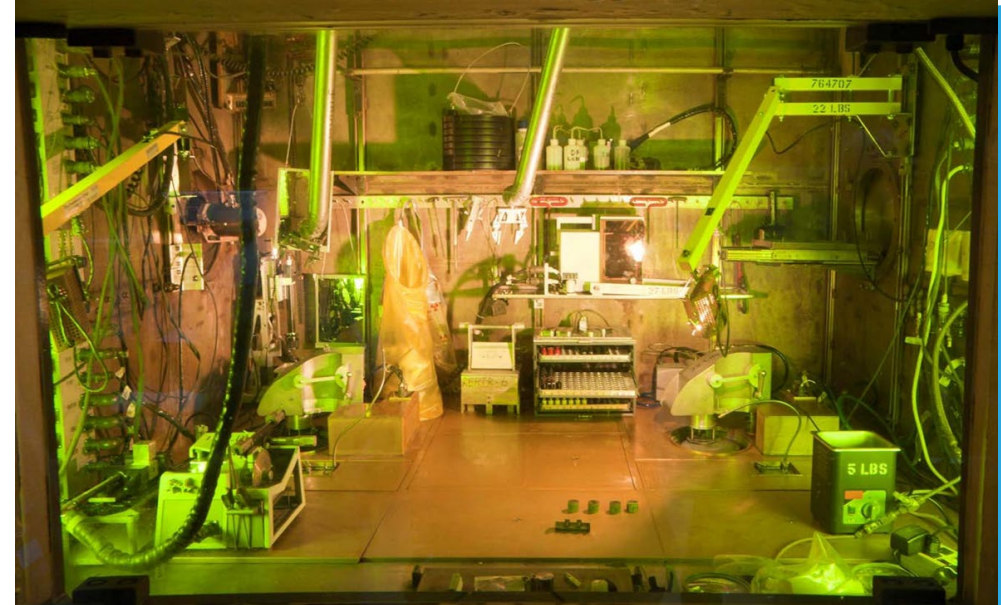
Facilities must be designed to handle radioactive materials (and contamination)

- Remote handled
  - Hot-Cells
  - Shielded enclosures
- Contact handled



# Hotcells

- Thick concrete walls
- Lead glass windows for shielding and visual
- Use manipulators to handle materials (up to 3 meters away from actual samples)
  - Limited feel feedback
  - Dexterity
  - Challenges with view potentially
- Inert vs Air Environments
- Increased training time to become proficient



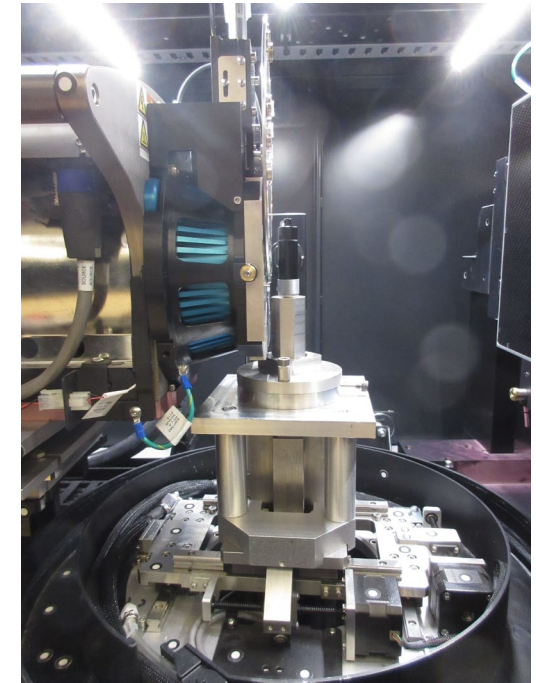
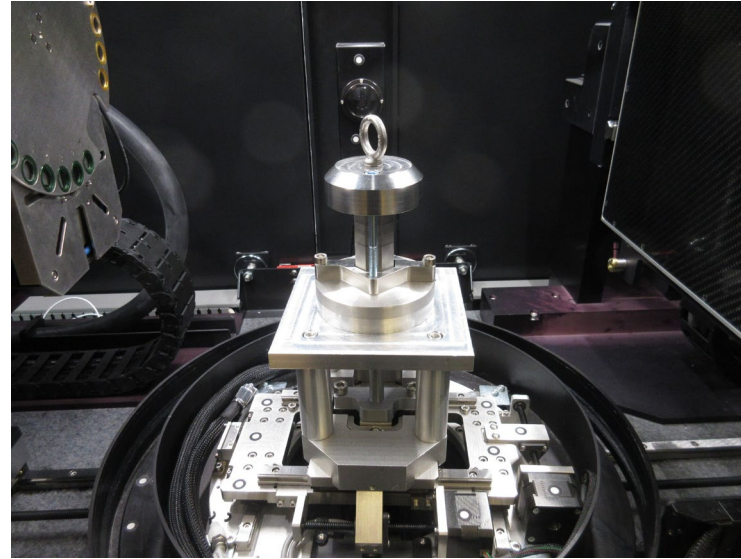


# Shielded Enclosures



# Gloveboxes/Hoods

- Glovebox
  - Full contamination control with potential for some radiation shielding
  - Limited dose rates for contact handling
  - Limited dexterity and feel
  - Use of shielding items to handle samples
- Hoods
  - Limited contamination and radiation protection
  - Samples of sample dose rates as gloveboxes
  - Better dexterity and feel
  - Increased contamination



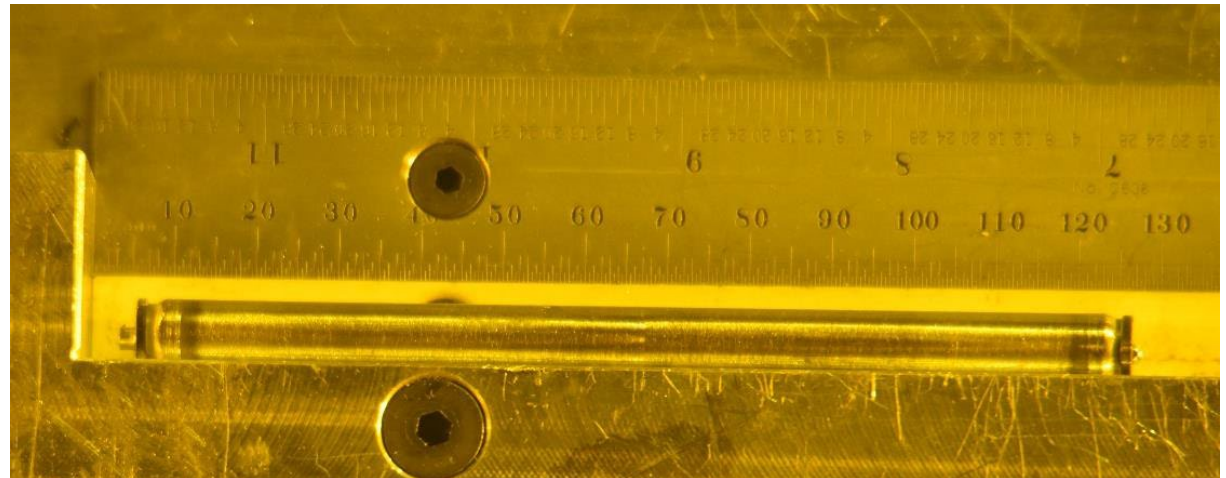
Push pop shielding for X-ray microscope testing





# Non-Destructive examination

- First step in the process
  - Visual examination
  - Thickness, diameter and oxide thickness measurements
  - Neutron radiography
  - Gamma scanning
  - Density
  - Fission gas analysis
  - X-Ray Micro-CT\*

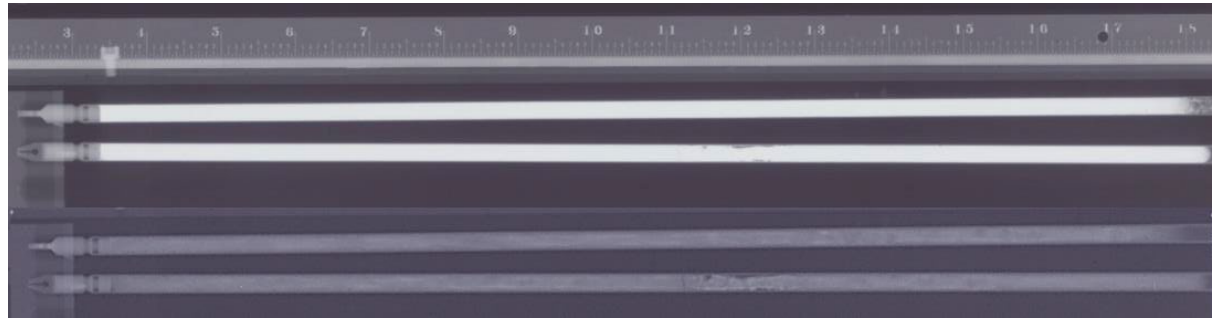






# Destructive Examinations

- NDE data must be examined before DE can start
- NDE data is used to determine sampling locations

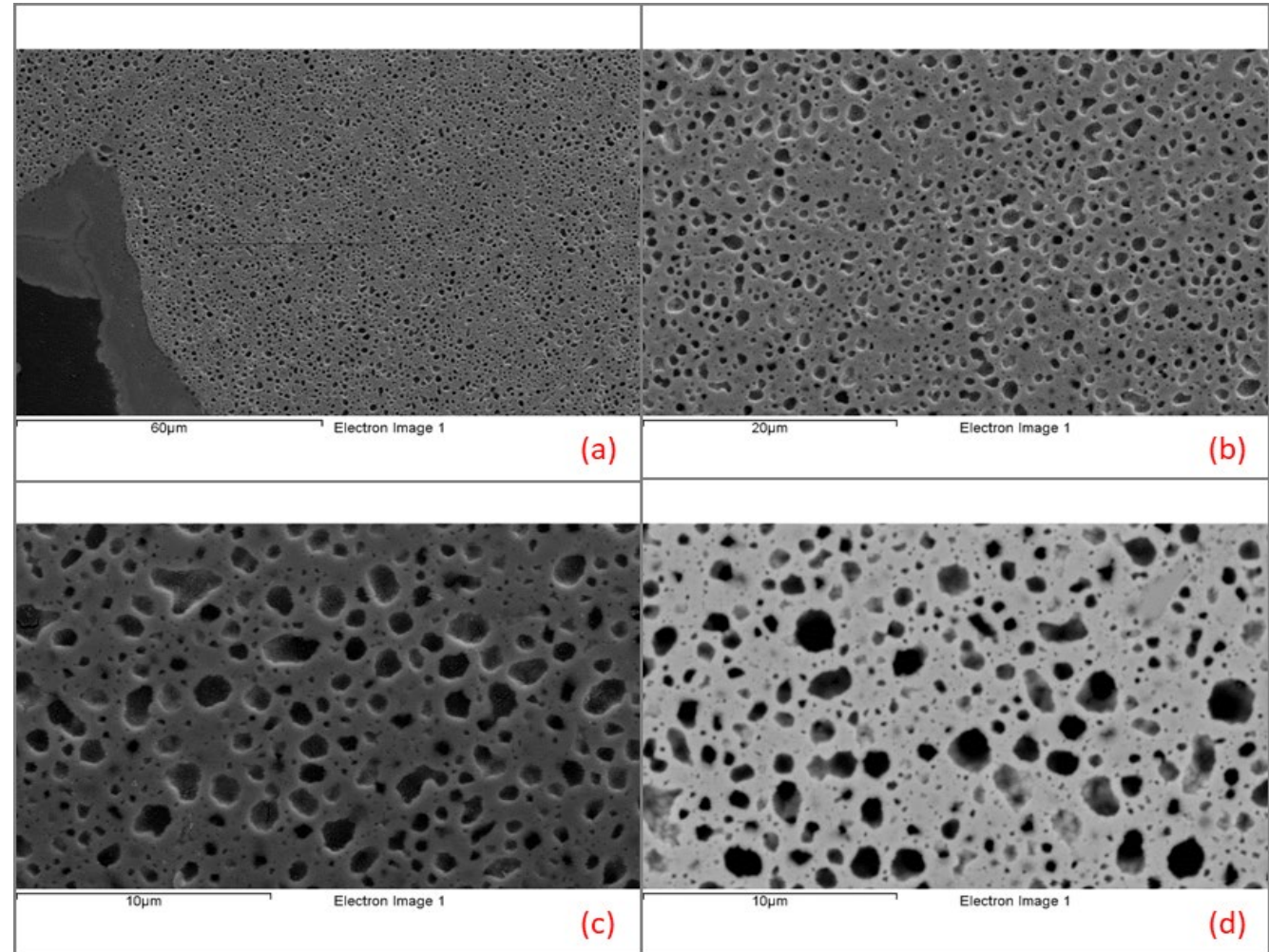


# Destructive Examinations

- Sampling and sample preparation
- Optical metallography
- Scanning electron microscopy
- Electron probe microanalysis
- Transmission electron microscopy
- X-ray diffraction
- Radiochemical burnup determination
- Thermophysical properties measurements
- Secondary ion mass spectrometry
- Neutron diffraction
- Atom probe tomography
- X-Ray Micro-CT\*

# Sample Preparation

- High quality prep challenging with non-radioactive samples
- Provides accurate and reliable results for advanced characterization results
  - Best practices provide the best results
- Compounded significantly with radiation and contamination fields added in with radiological samples

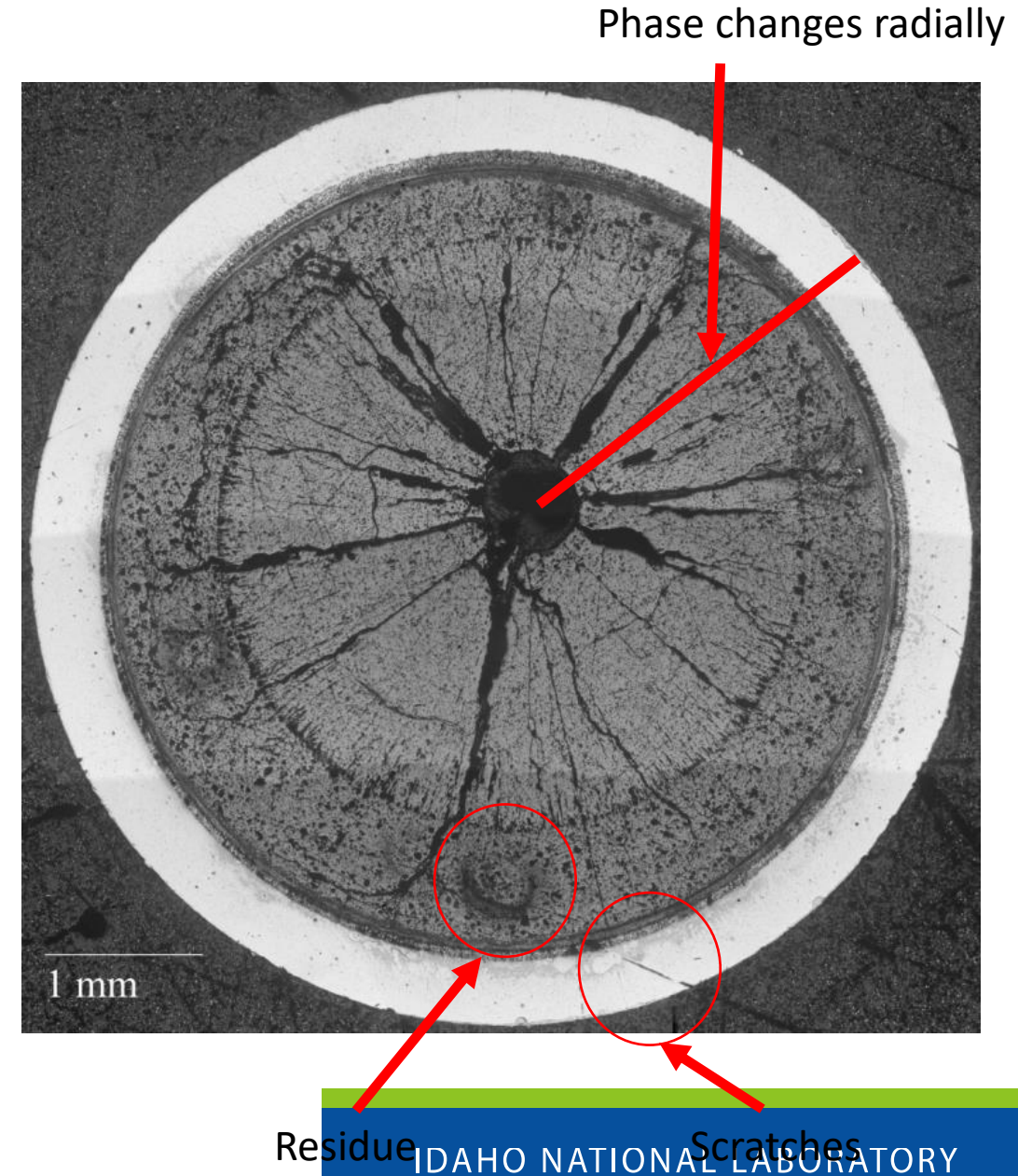


Porosity of U-Mo fuel



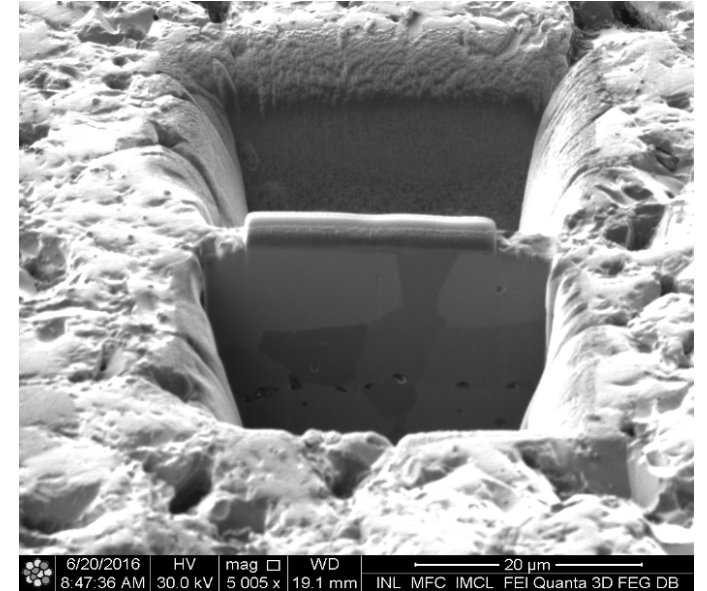
# Example: U-Zr Based Fuel Cross-Section

- U-Zr based fuel in a HT-9 cladding
- Multi-phases in fuel
  - Polish different rates (rounding)
  - Holds liquid and could lead to residues
  - Increased polish times of different phases
- Porosity
  - Buildup of polishing debris in pores which lead to scratching later
- Residue staining in areas from cleaning
- Removal of Cs-bearing precipitates due to use of water
- Radiolysis of the epoxy due to radiation fields
- Decontamination of the material leads to increased handling
- Oxidation of the surface if perform in air vs inert
- Many more!

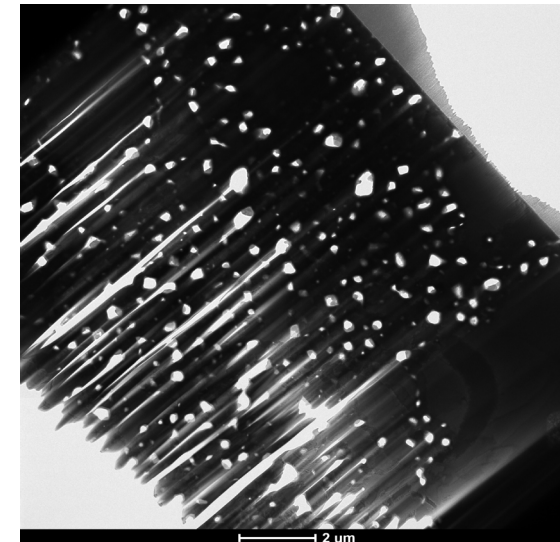


# Focused Ion Beam Microscopy

- Site Specific TEM and APT sample prep
- Utilize FIB to remove defects on the surface of the material
  - Mechanical damage from polishing
  - Oxidation
  - Can get electron backscatter diffraction quality samples with some materials
- Drawbacks
  - Curtaining effects
  - Sputtering of radioactive materials into the instrument

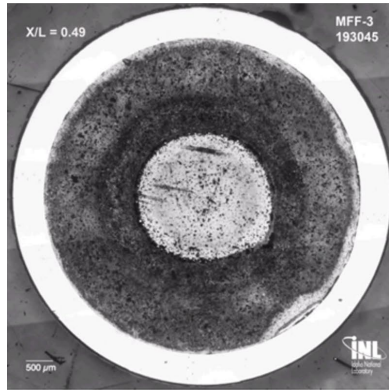


Cross section of (U,Pu)O<sub>2</sub> Fuel

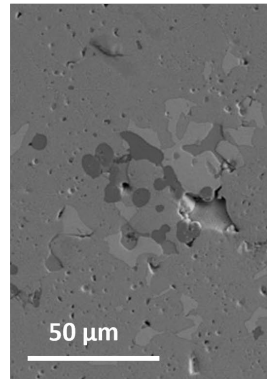
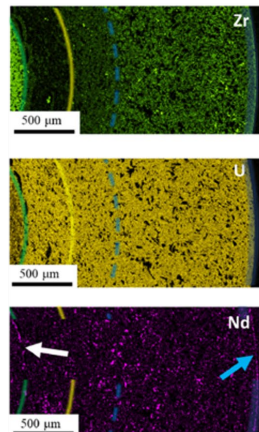
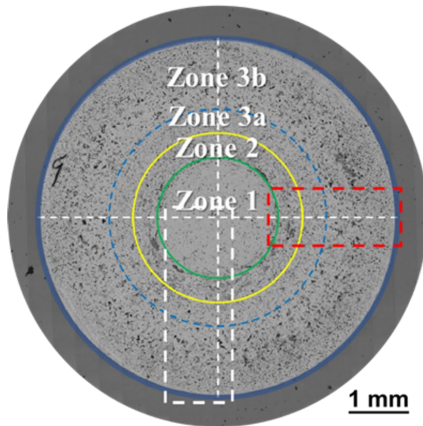




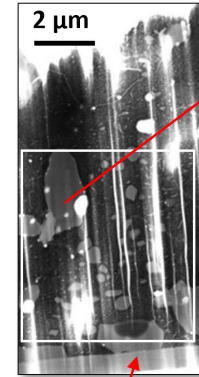
# PIE Example Workflow



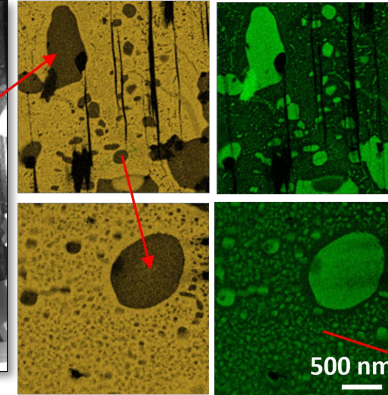
Optical microscopic examination  
(U-10Zr fuel)  
(1mm,  $10^{-3}$  m)



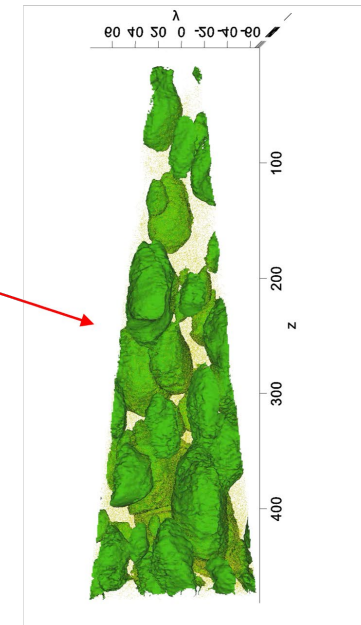
Scanning electron microscopy characterization  
of U-10Zr grain and precipitate (1  $\mu\text{m}$ ,  $10^{-6}$  m)



U Zr



Transmission electron  
microscopy identification of  
Zr nano precipitate  
(2-5 nm,  $10^{-9}$  m)



Atom probe tomography study of  
Zr atom distribution in 3D  
(3  $\text{\AA}$ ,  $10^{-10}$  m)

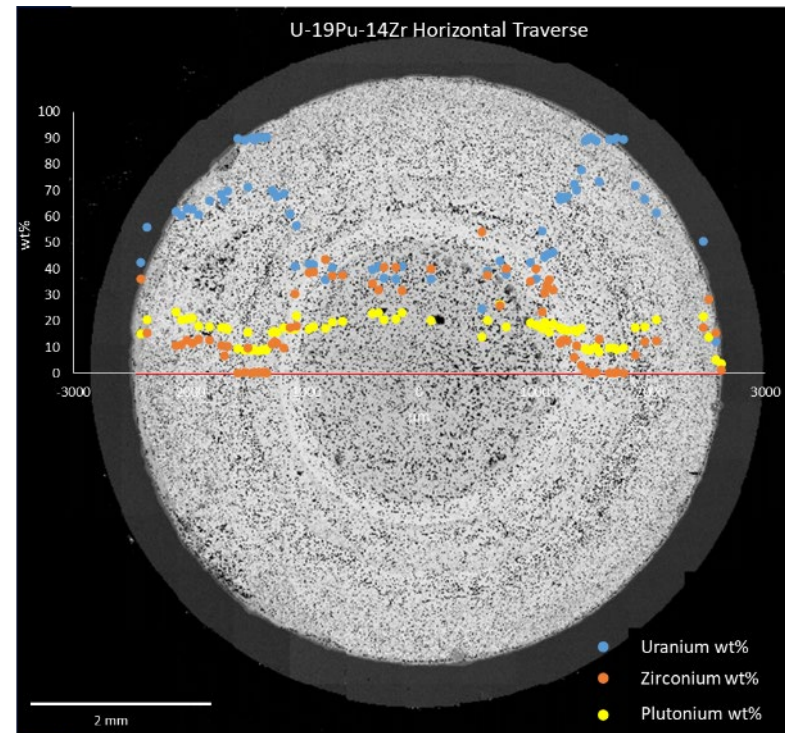
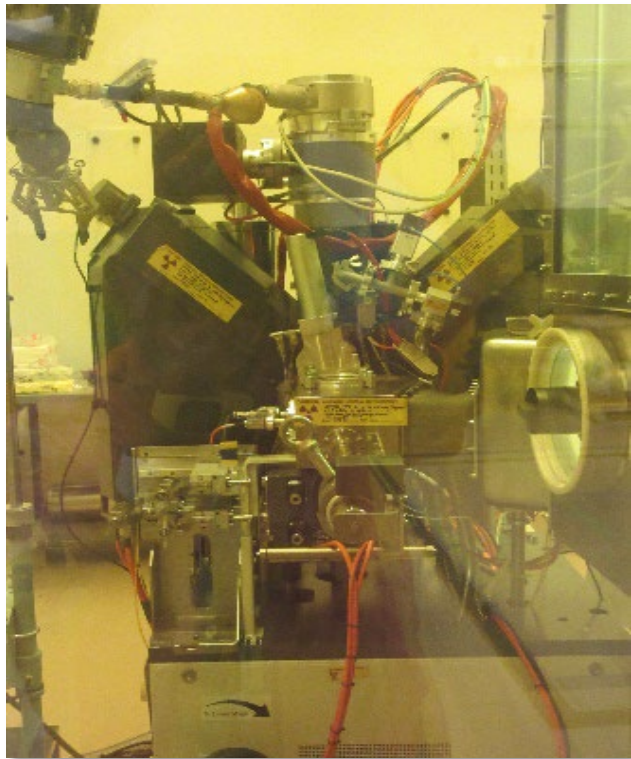
PIE microstructure characterization covers 7 orders of magnitude



# Electron Probe Microanalyzer (EPMA)

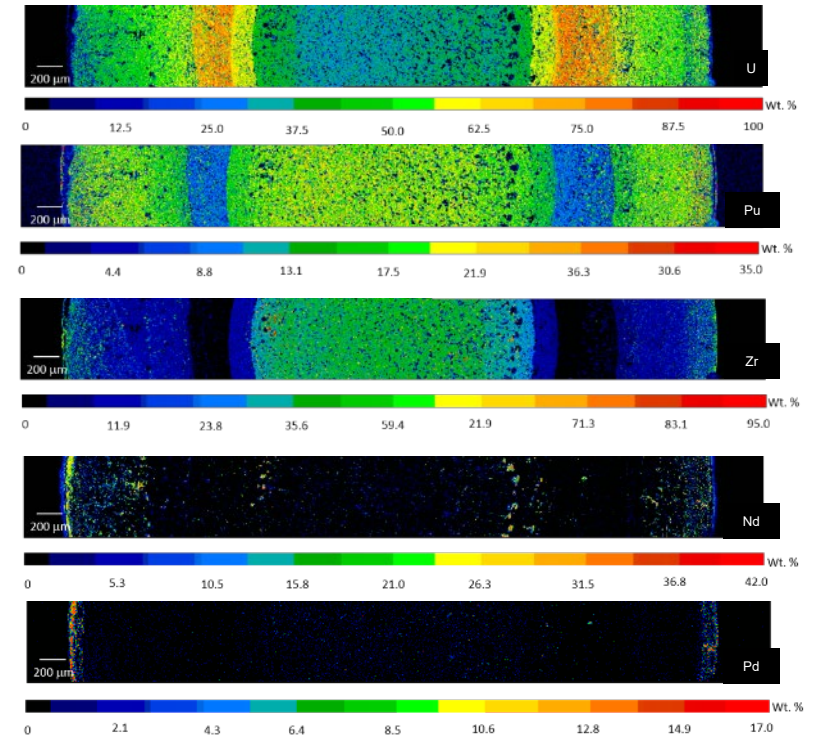
- Shielded to 3 curies of Cs-137 radiation energy allowing analysis of irradiated fuel pin cross sections
- Capable of quantitative analysis of solid specimens on a micrometer spatial scale
- Can detect elements from B-Cm+ (including gasses trapped in bubbles)

## Quantitative analytical diameter traverse of mapped diameter region of U-19Pu-14Zr



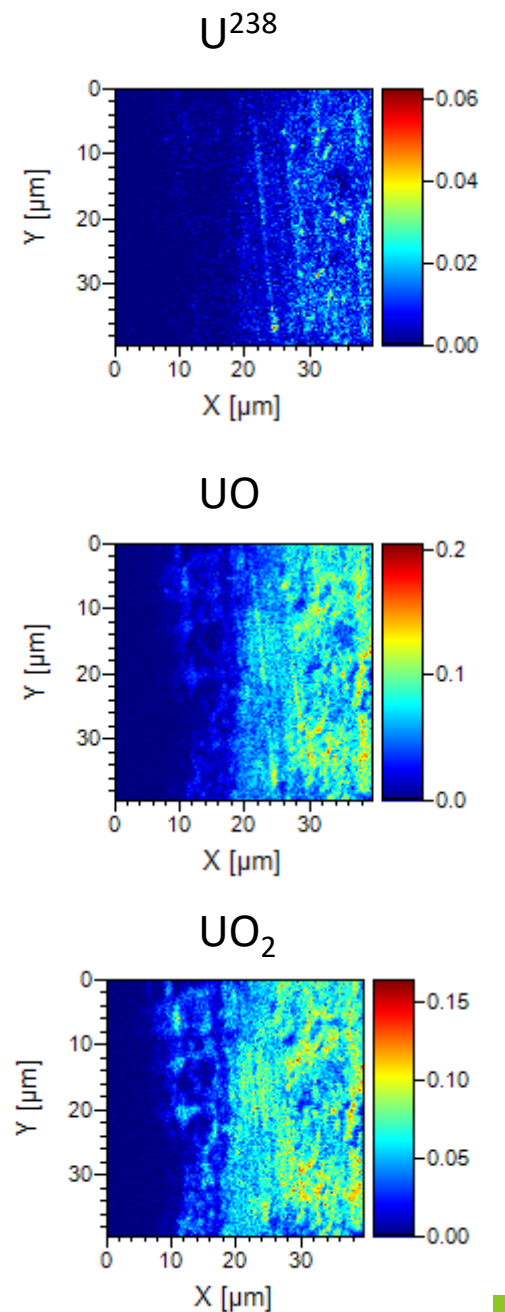
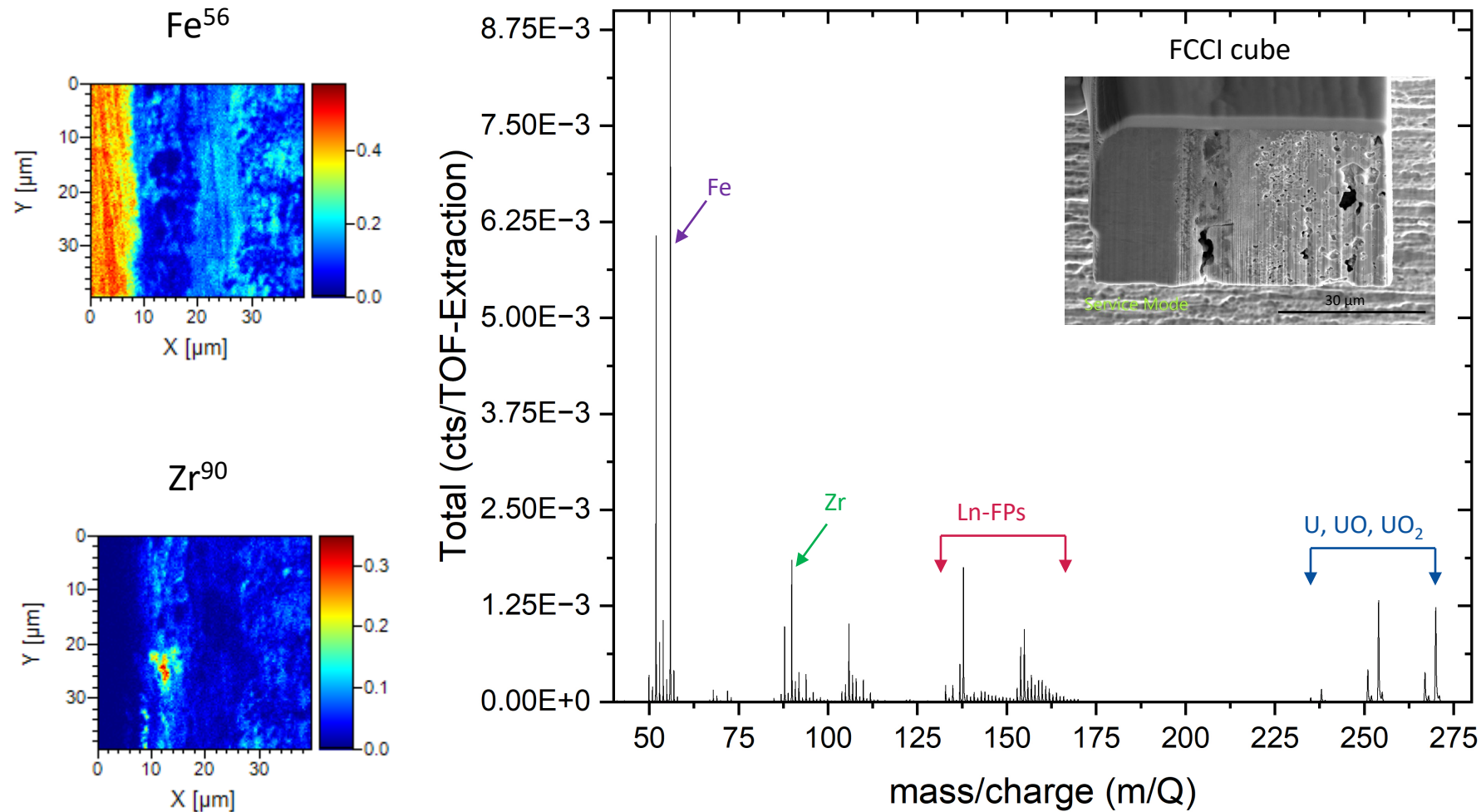
## Quantitative X-ray maps of irradiated U-19Pu-14Zr

- Enhanced center pin Zr concentration
- Depleted center pin U concentration
- Rare earth element phases near center
- Asymmetrical fuel-cladding chemical interaction



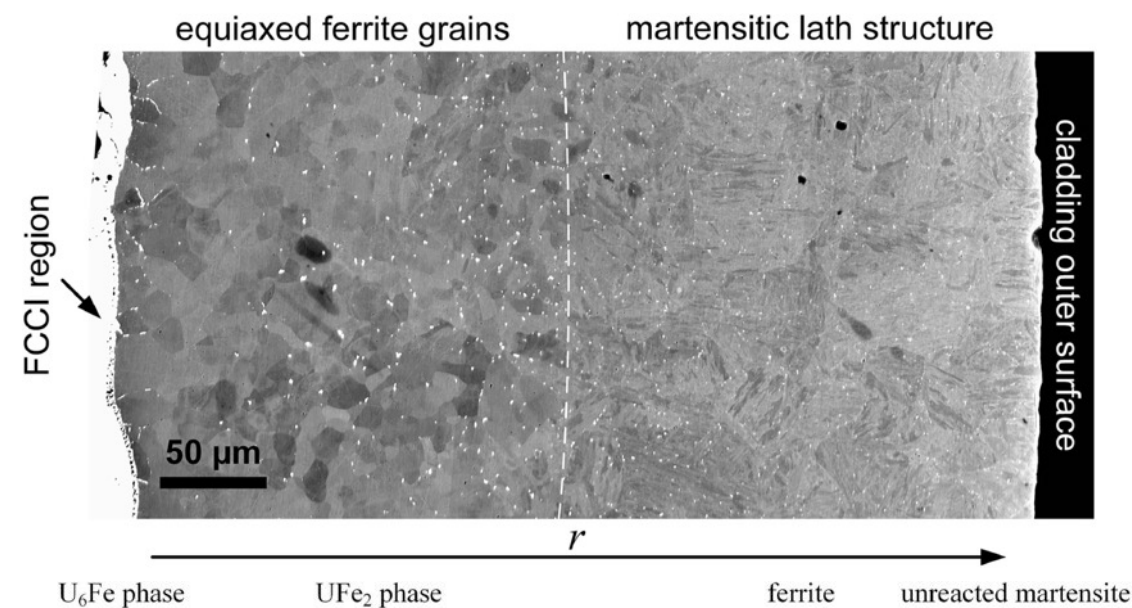
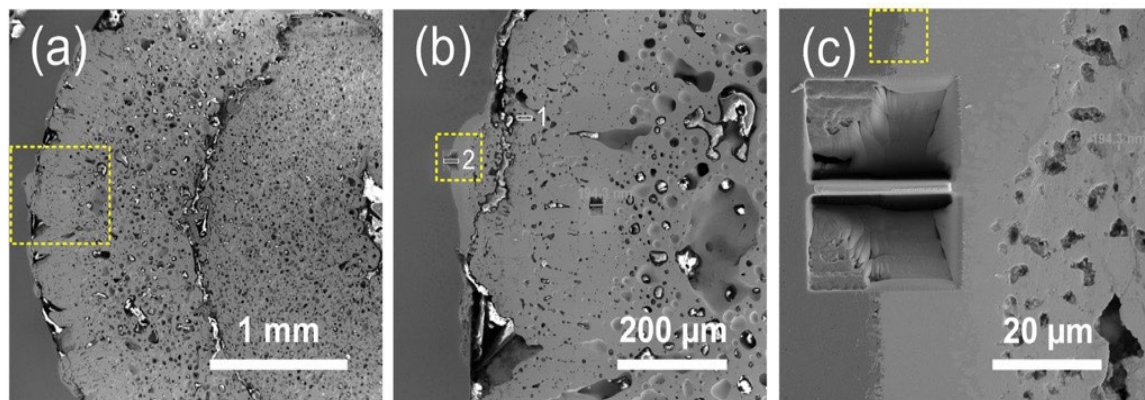
# Secondary Ion Mass Spectrometry

Mass spectrum

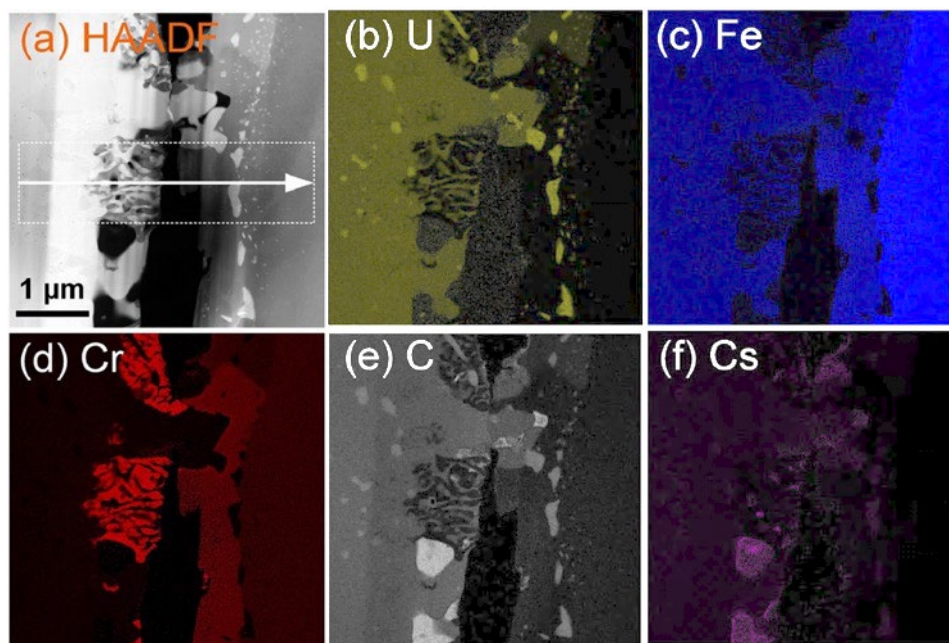
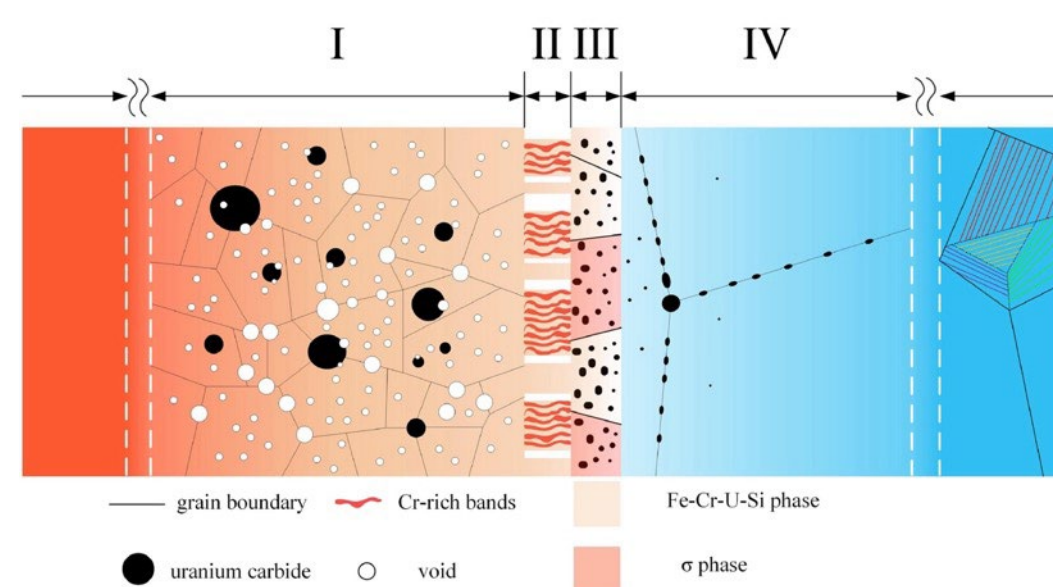




# TEM analysis

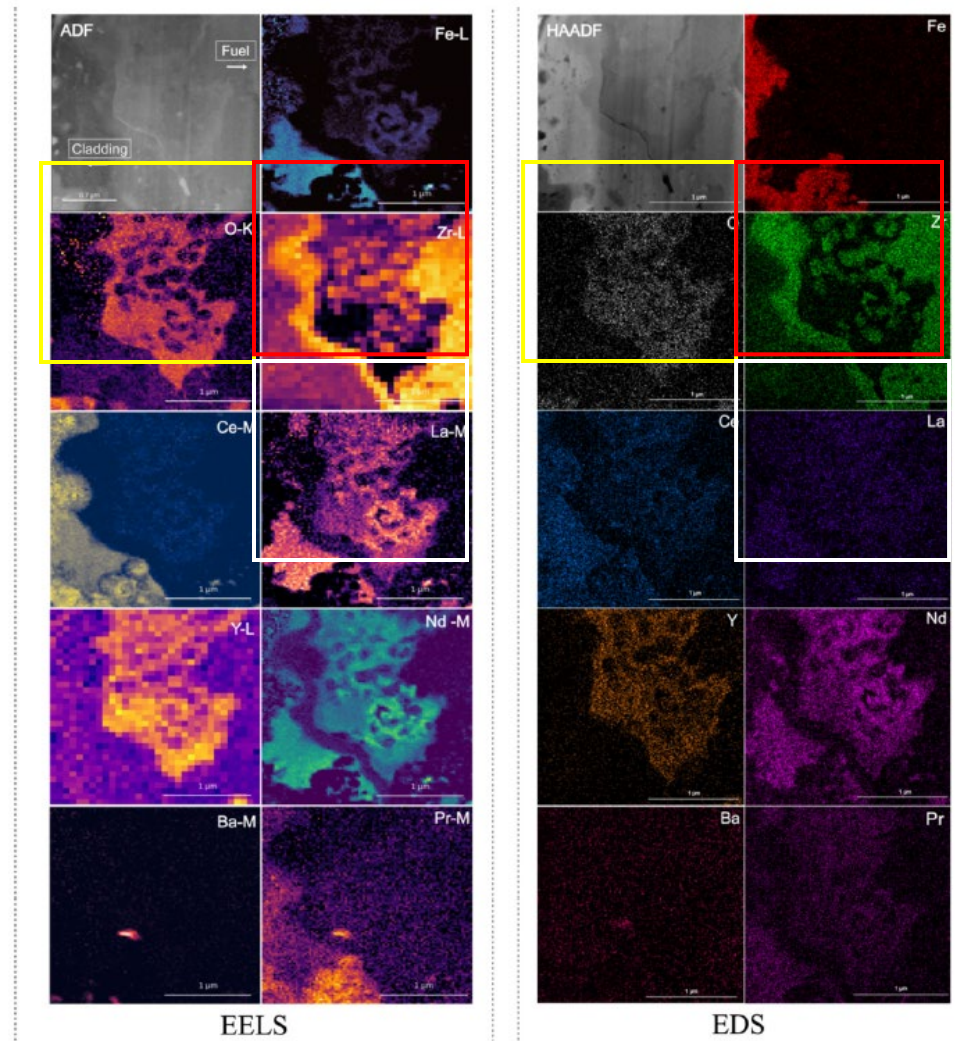


$r$   
U<sub>6</sub>Fe phase      UFe<sub>2</sub> phase      ferrite      unreacted martensite



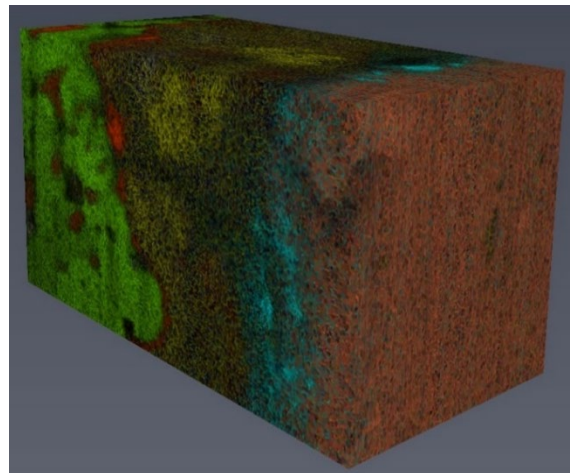
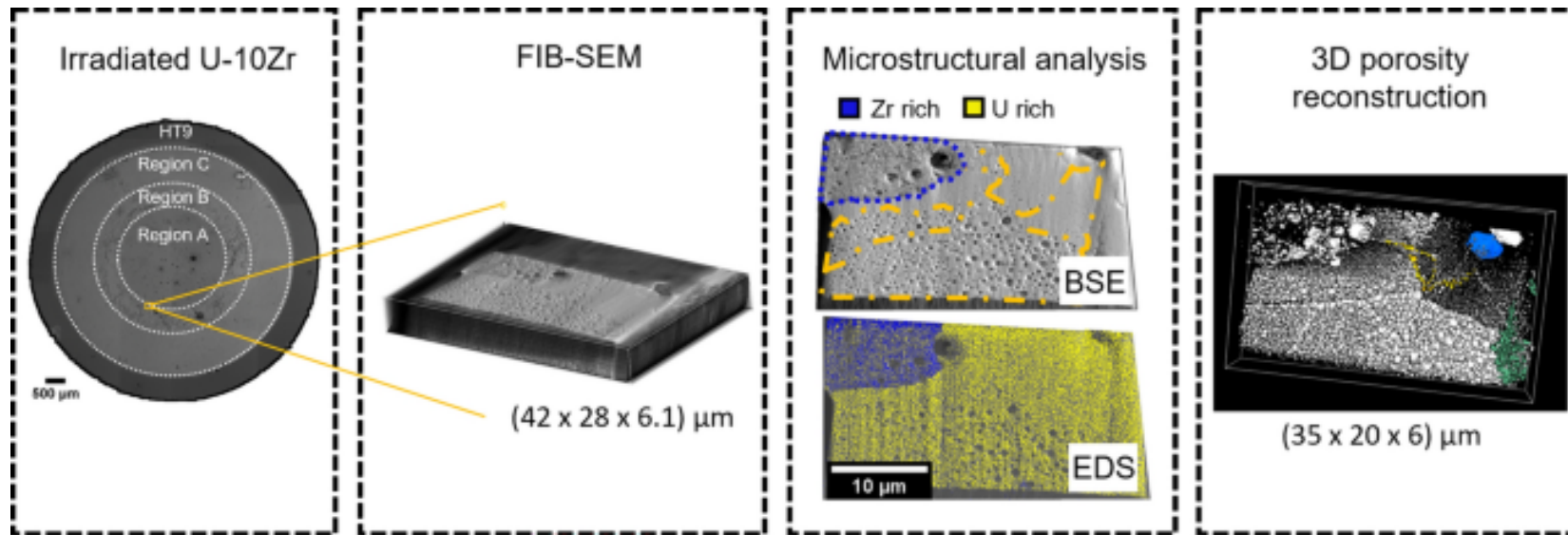


# EELS and EDS



**Fig. 5.** Elemental maps of a FCCI region from a high burnup fuel (13.2 at%) along with their microstructure image through STEM-EELS (left) and EDS (right) (Pradhan *et. al*).

# 3D FIB/SEM reconstructions



# Thermal diffusivity measurements

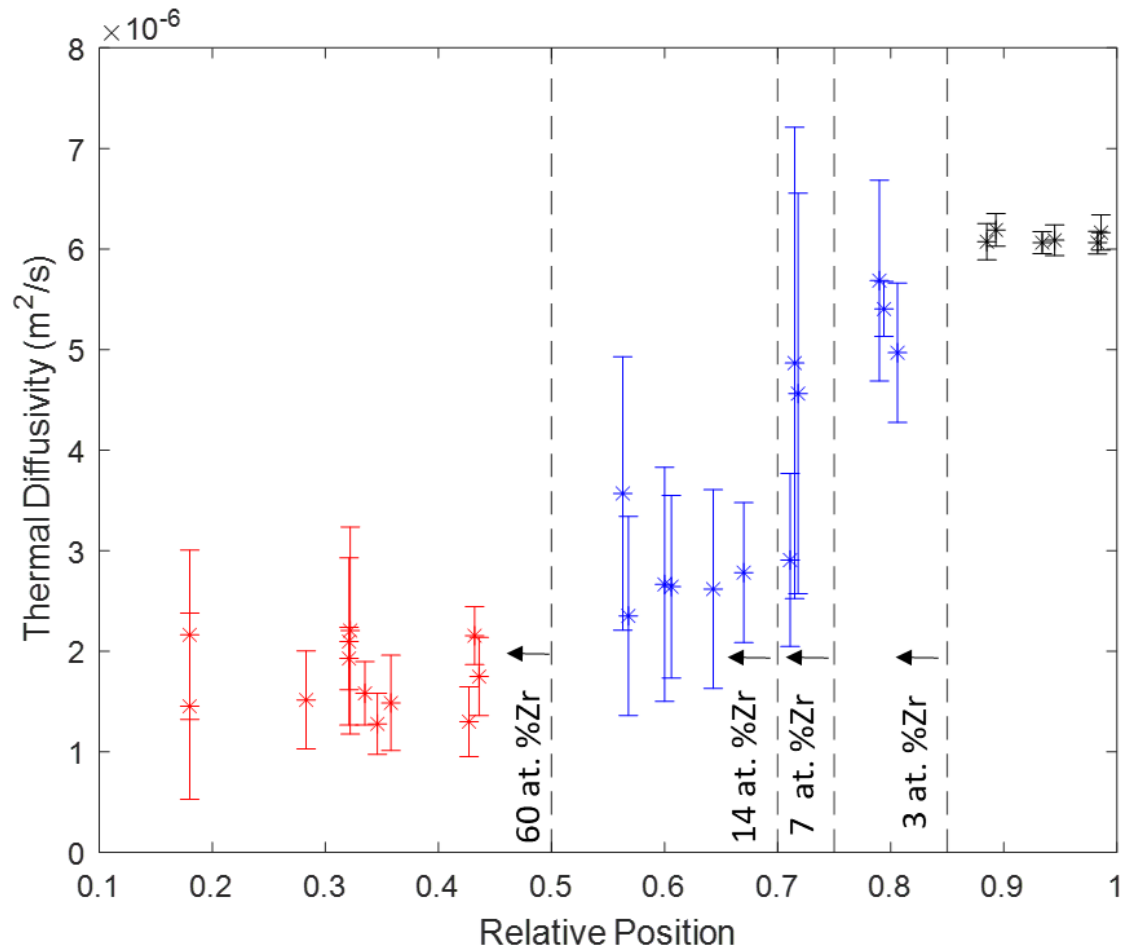
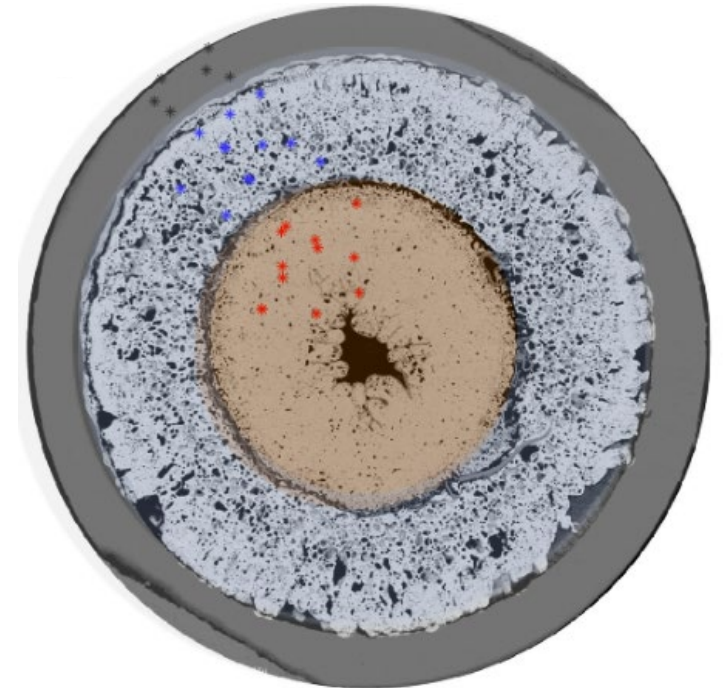


Figure 2 Thermal diffusivity values from TCM measurements of the inner fuel region (red), annular fuel region (blue), and cladding region (black).







# Idaho National Laboratory

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